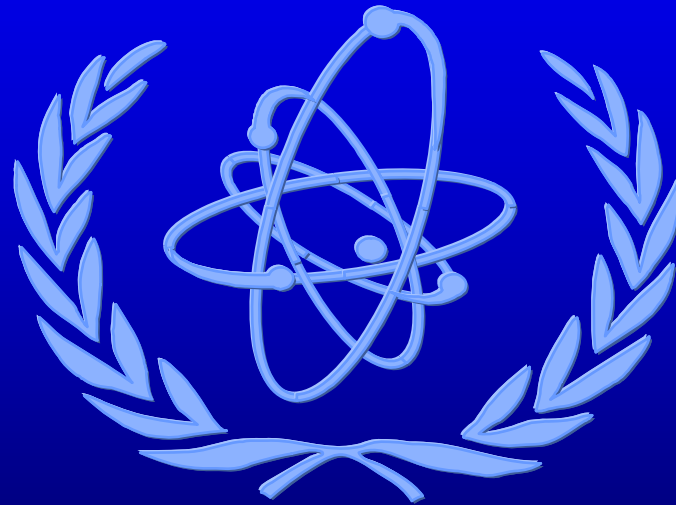


MODULE 6: Level 1 – Level 2 Interface



Outline of Discussion

- Objectives of the Level 1-2 Interface
- Technical Approach
- Examples



Objectives: Level 1-2 Interface

1. **Add containment system response to probabilistic assessment of accident definitions**
 - **Level 1 models do not consider containment systems that are not required to prevent core damage**
 - **Containment response strongly depends on availability of safeguard systems**
-



Objectives: Level 1-2 Interface

2. **Identify the minimum number of unique accident scenarios to be addressed in Level 2 analysis**
 - **The number of accident scenarios should be 'small' to make deterministic analysis practical.**
 - **Many accident sequences considered in a Level 1 PSA would result in similar accident progression and containment response**
-



Technical Approach

- **Objective-1: Add containment systems**
⇒ **Containment safeguards tree or bridge tree**
 - **Objective-2: Minimize number of accident scenarios**
⇒ **Plant Damage State (PDS) analysis**
-

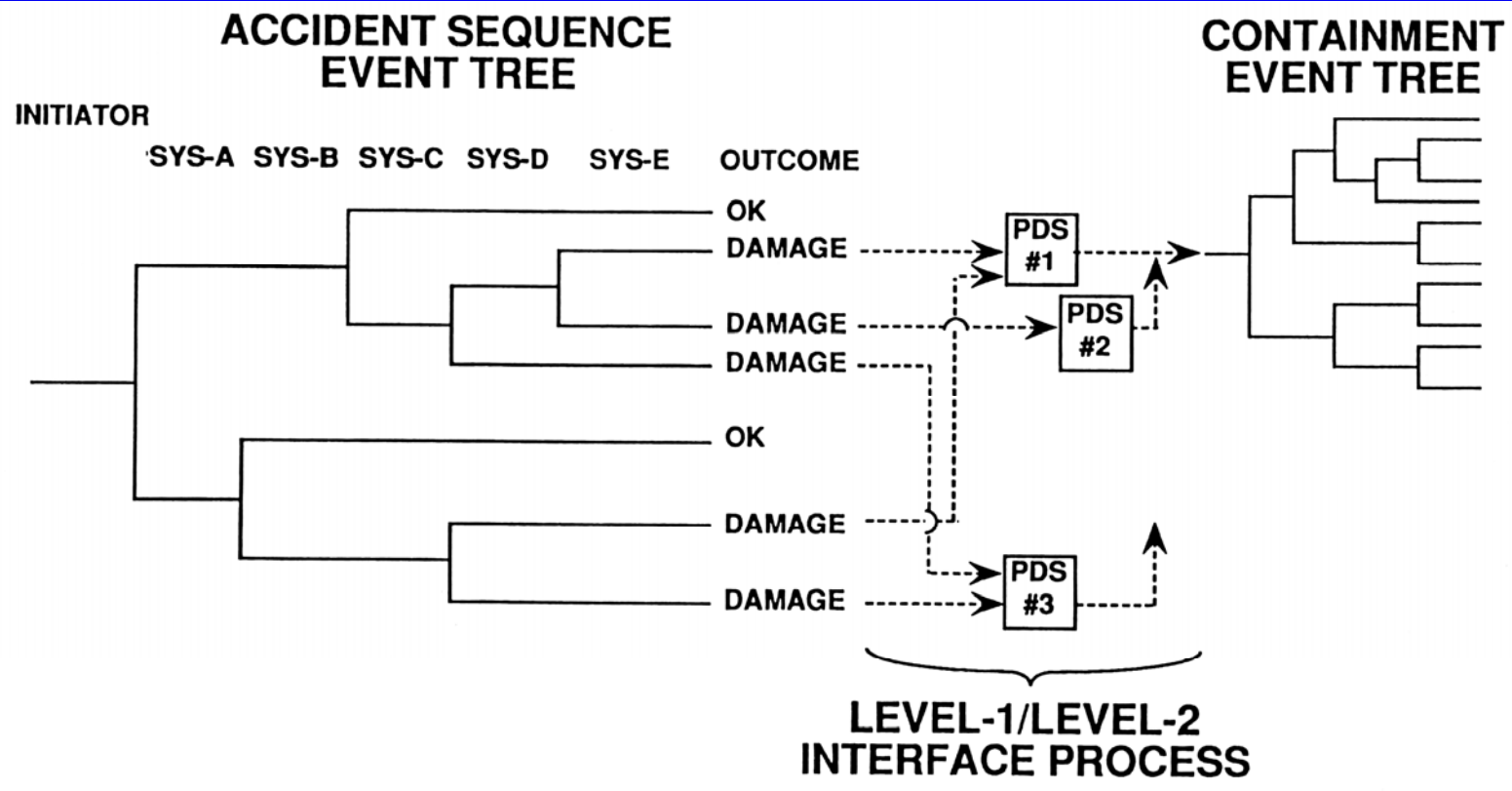


PDS Analysis

- **Procedure: Sort core damage sequences into groups or 'bins' that represent common features of severe accident progression:**
 - **Accident timing**
 - **Release pathways for fission products to containment**
 - **Energy imparted to containment**
- **Each PDS represents a unique set of initial and boundary conditions to the analysis of containment response**
-



PDS Binning Process



Sorting or Binning Criteria Define Status of Barriers to F.P. Release

Barriers	Systems
Fuel clad	Reactor power Core cooling systems
Reactor vessel	Safety/relief valves Reactor coolant system pressure boundary
Containment	Level of isolation Heat removal Water sources
Retention capabilities	Spray systems Pool/sump water
Secondary/Auxiliary building	Ventilation filters Building integrity



Simple Example

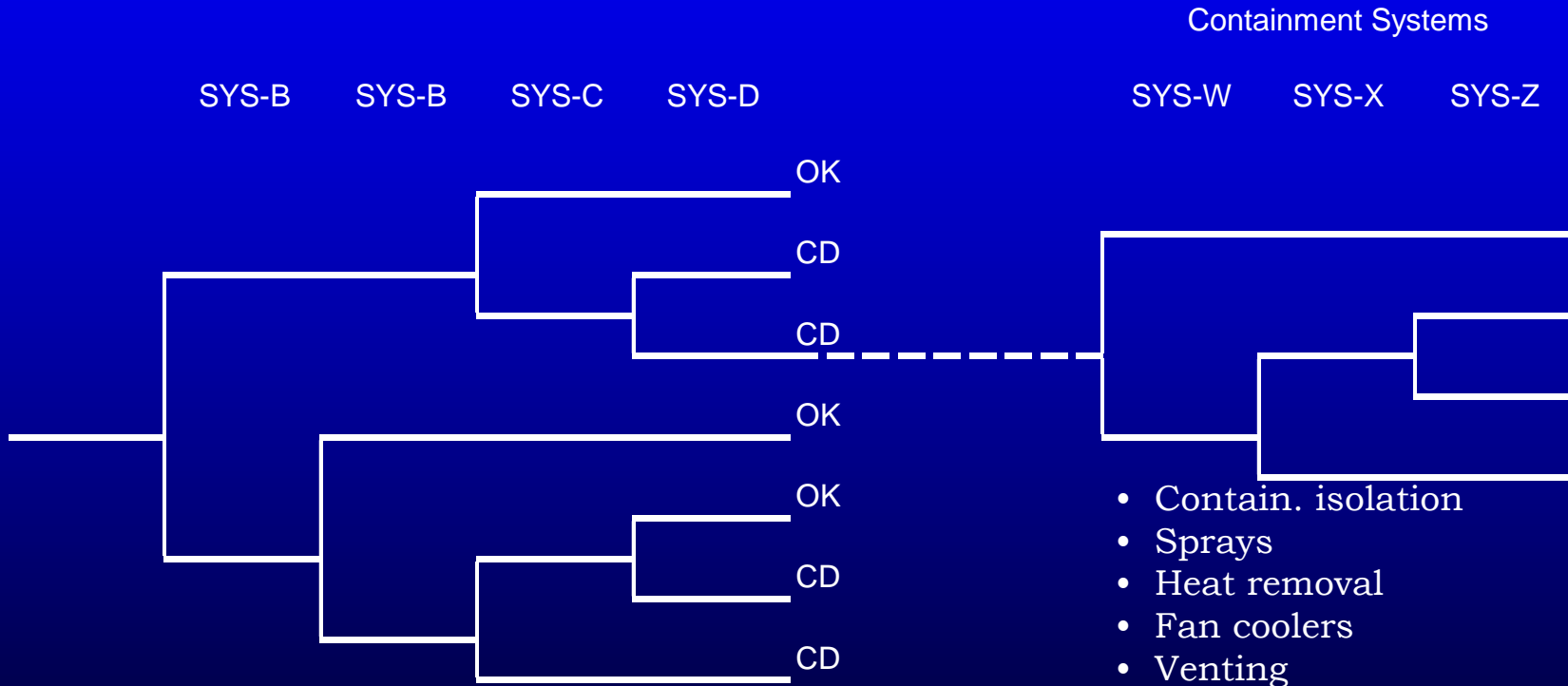
- Sequence 1: Transient, No coolant makeup to reactor vessel
 - Cut Set 1A: T1 * injection valves (T1 = LOSP)
 - Cut Set 1B: T2 * pumps
 - Cut Set 1C: T1 * diesel generators
- Sequence 2: Transient, Coolant injection fails in long-term
 - Cut Set 2A: T1 * heat removal * containment
 - Cut Set 2B: T2 * heat removal * (vent) * pumps cavitate
 - Cut Set 2C: T2 * pump cooling

Cut Set	Coolant Injection	Contain. Isolation	Containment Spray	PDS
1A	Fail	OK	OK	1
1B	Fail	OK	Fail	2
1C	Fail – recoverable	OK	Fail – recoverable	2'
2A	Fail	Fail	Fail	3
2B	Fail	Vent	Fail	4
2C	Fail	OK	Fail	2



Containment Safeguards Tree

- Extend Level 1 sequence event trees to include containment systems



Examples from Past PSAs

- PDS results presented in several forms
 - Graphical (event tree format)
 - Tabular (matrix)
 - Character string (vector format)
- Content of various forms is the same
 - Only a preference in presentation



Graphical format

Entry from Level 1 Sequence with PDS Information	Containment Bypass	Integrity of Primary System Pressure Boundary	Availability of ac Power	Time of Power Recovery (if lost before core melt)	Containment Sprays Long Term Availability	Long Term Injection Status	Pressure in Reactor Vessel at Core Melt at Vessel Failure	PDS #
					yes	yes	very high	1
				before containment fails	no	no	very high	2
			lost before core melt		no	no	very high	3
				no recovery	no	no	very high	4
		intact			yes	yes	very high	5
			on		no	no	very high	6
					no	no	very high	7
		very small break	on		yes	yes	very high	8
					no	no	very high	9
					no	no	very high	10
	no bypass	small break	on		yes	yes	high	11
					no	no	high	12
		medium break	on		yes	yes	medium	14
					no	no	medium	15
					no	no	medium	16
		large break	on		yes	yes	low	17
					no	no	low	18
		vessel rupture	on		yes	yes	low	19
								20
CDF	ISLOCA	ISLOCA	on					21
	SGTR	SGTR	on					22
	SGTR+SORV	SGTR	on					23



Tabular format

Conditions at the time of vessel breach		Primary Containment Intact at Time of Core Melt?																			
		Yes										No									
		Secondary Building Isolated?										Secondary Building Isolated?									
		Yes					No					Yes		No							
		Ventilation OK?					No (C)					Ventilation OK?		No (F)							
		Yes (A)					No (B)					Yes (D)		No (E)		No (F)					
Vessel pressure	Water on Contain. Floor?	Water to Core Debris?				Water to Core Debris?				Water to Core Debris?				H ₂ O to debris?		H ₂ O to debris?		H ₂ O to debris?			
		Yes		No		Yes		No		Yes		No		Yes	No	Yes	No	Yes	No		
		Containment cooled?		No		Containment Cooled?		No		Containment cooled?		No									
		Yes (I)	No		Vented?		Yes (I)	No		Vented?		Yes (I)	No		Vented?						
			Yes (J)	No (K)	Yes (L)	No (M)		Yes (J)	No (K)	Yes (L)	No (M)		Yes (J)	No (K)	Yes (L)	No (M)	(K)	(M)	(K)	(M)	(K)
		High	--																		
Low	Wet																				



Character string format

H.P. Station Blackout:
T1-B2-X

SLOCA without heat
removal:
S2-U2-W

	Attribute	State Identifier	State Description
(1)	Initiating event	A S1 S2 V T1 T2 T3	LBLOCA: Large ¹ LOCA IBLOCA: Intermediate LOCA ² SBLOCA: Small ³ LOCA LOCA outside containment TLOP or PLOP: Loss of offsite power SORV: Stuck-open SRV/PRV Other transients
(2)	Reactor status	-- C	Scram; no fission power Power above decay heat (no scram)
(3)	Electric Power status	-- B1 B2	Electric power available Onsite ac power failed, dc OK Onsite ac and dc power failed
(4)	Vessel pressure at onset of core damage	-- X	Vessel depressurized ⁵ Vessel at high pressure ⁶
(5)	High pressure coolant injection status	-- U1 U2	HPI available as required HPI operates until batteries exhausted HPI not available
(6)	Low pressure coolant injection status	-- V	LPI available LPI not available
(7)	Containment heat removal status	-- W	RHR available RHR not available
(8)	Containment	-- Y1 Y2	Intact (design basis leakage only) Failure of containment isolation Containment venting successful



Dependencies

- **Containment safeguards tree(s) must be directly coupled to Level 1 sequence events trees to properly handle dependencies**
 - **Shared components**
 - **Common support systems**
 - **Prior human actions**



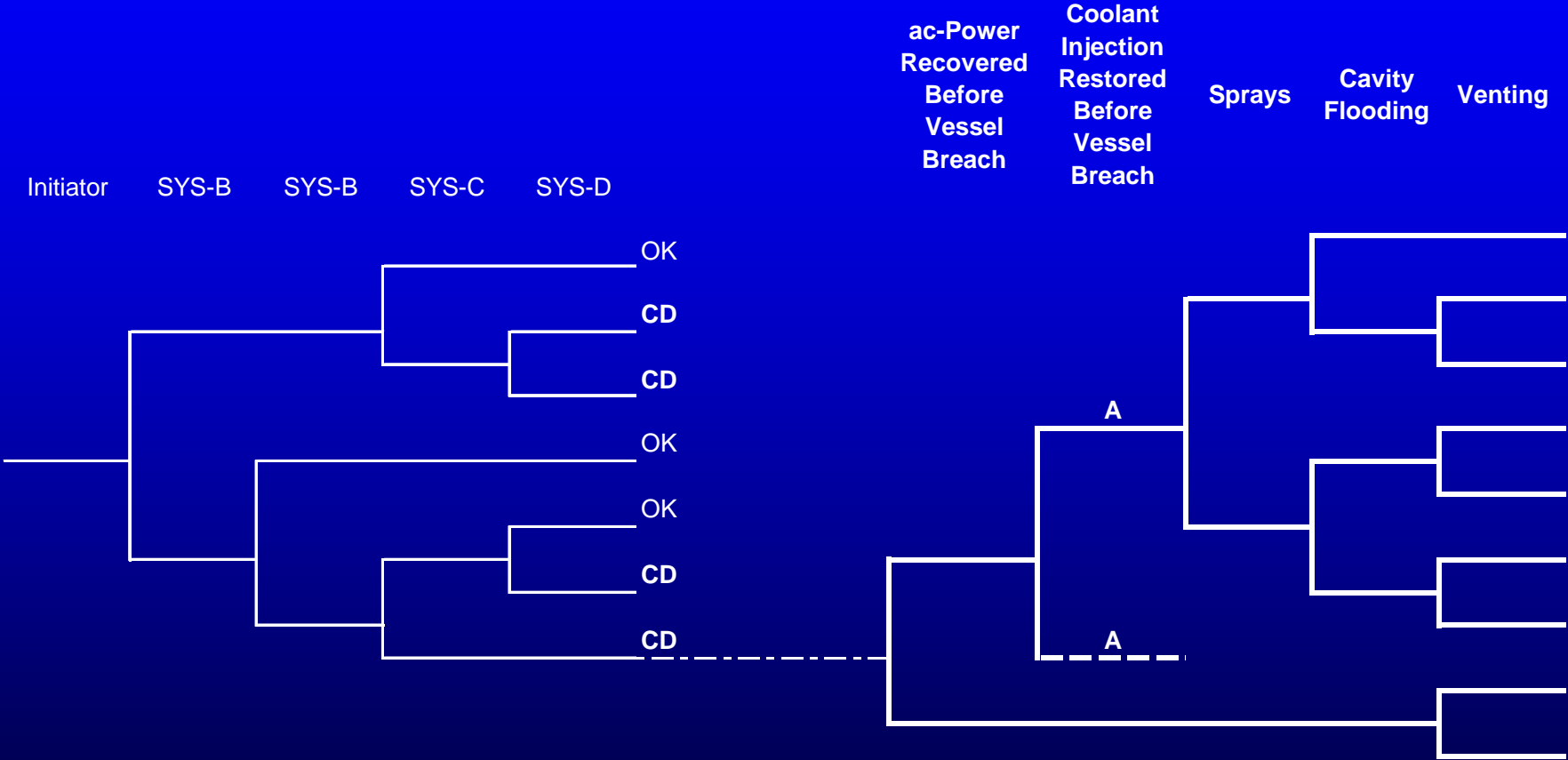
An Optional Extension

(Containment Safeguards → Bridge Tree)

- **Extend period of accident recovery to vessel breach**
 - **Restoration of ac power after onset of core damage**
 - **Post-core damage operator actions (severe accident management guidelines)**
 - ❖ Reactor vessel depressurization
 - ❖ Reactor cavity flooding
 - ❖ Containment venting
 - **Requires some knowledge of timing of in-vessel core melt progression**
-



Example Bridge Tree

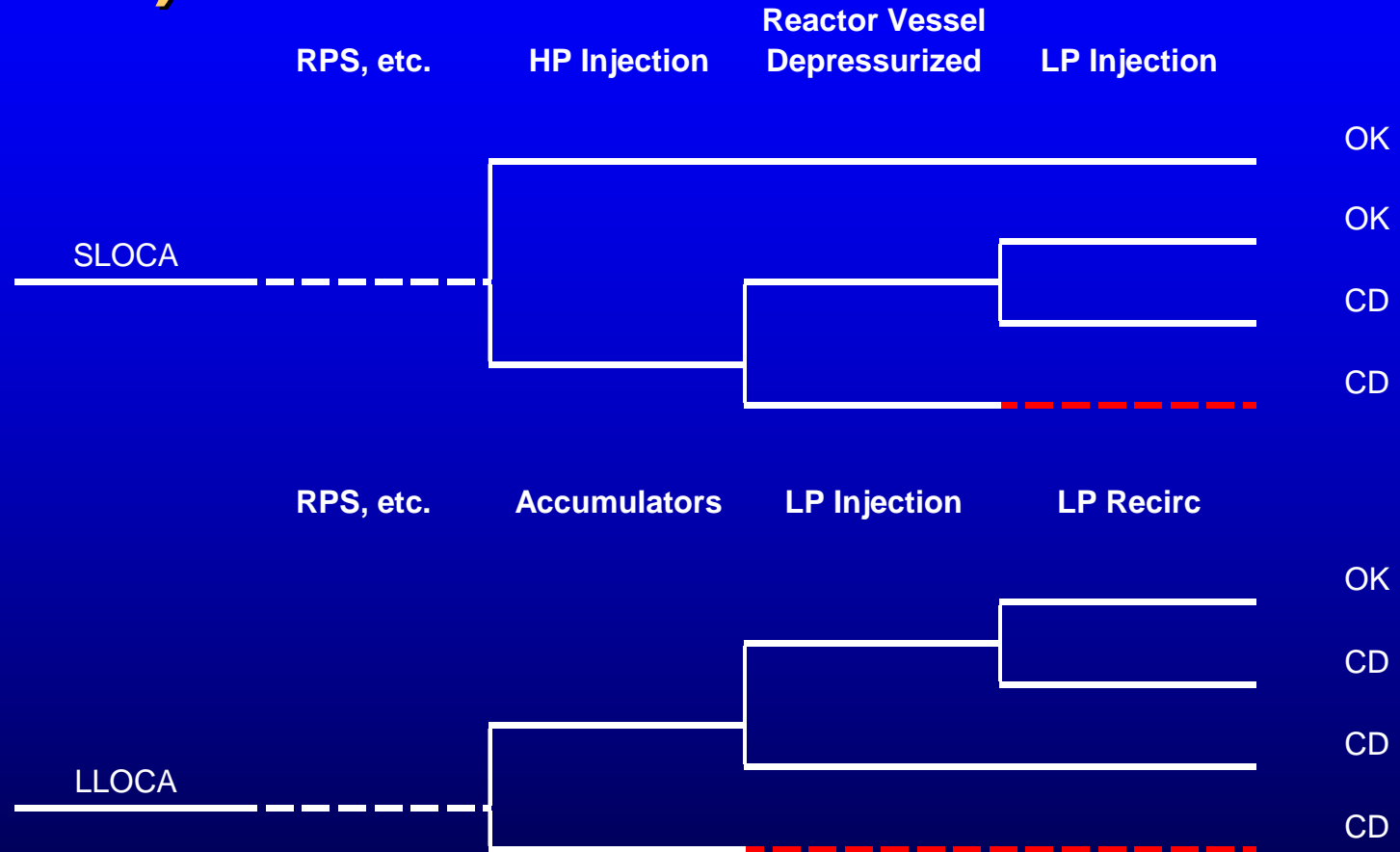


Final Issue: Resolution of the Status of 'Abandoned' Systems

- **The status of certain systems important to the Level 2 may be 'unknown' from the Level 1**
 - **Sequence event trees ignore branches at locations where success criteria have already been violated**
 - **Success or failure not decisive for preventing core damage, but can influence subsequent severe accident progression**
 - **Example: LOCA with failure of one part of injection success criteria**
-



Examples of Unresolved System Status (LP-ECCS)



Summary

- **Level 1 – Level 2 Interface is an analysis task**
 - **Couple Level 1 core damage sequence information with new information on containment system availability**
 - **Translate sequence cut sets to initial and boundary conditions for analysis of containment response**
 - **Supporting deterministic analysis (code calculations) may be necessary to properly group ‘similar’ accident sequences**
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